



## GCFR-PROTEUS EXPERIMENTAL PROGRAM CORE 11- NOMINAL CORE CONFIGURATION BENCHMARK

Gareth Newman | Graduate Student, Material Science Department, Nuclear Engineering Program

Kelly Jordan | Principle Investigator, Material Science Department, Nuclear Engineering Program

Gregory Perret | Collaborator, Paul Scherrer Institut

John Bess | Collaborator, Idaho National Lab

# Special Thanks

- This project was supported by the U.S. Department of Energy, NEUP program.

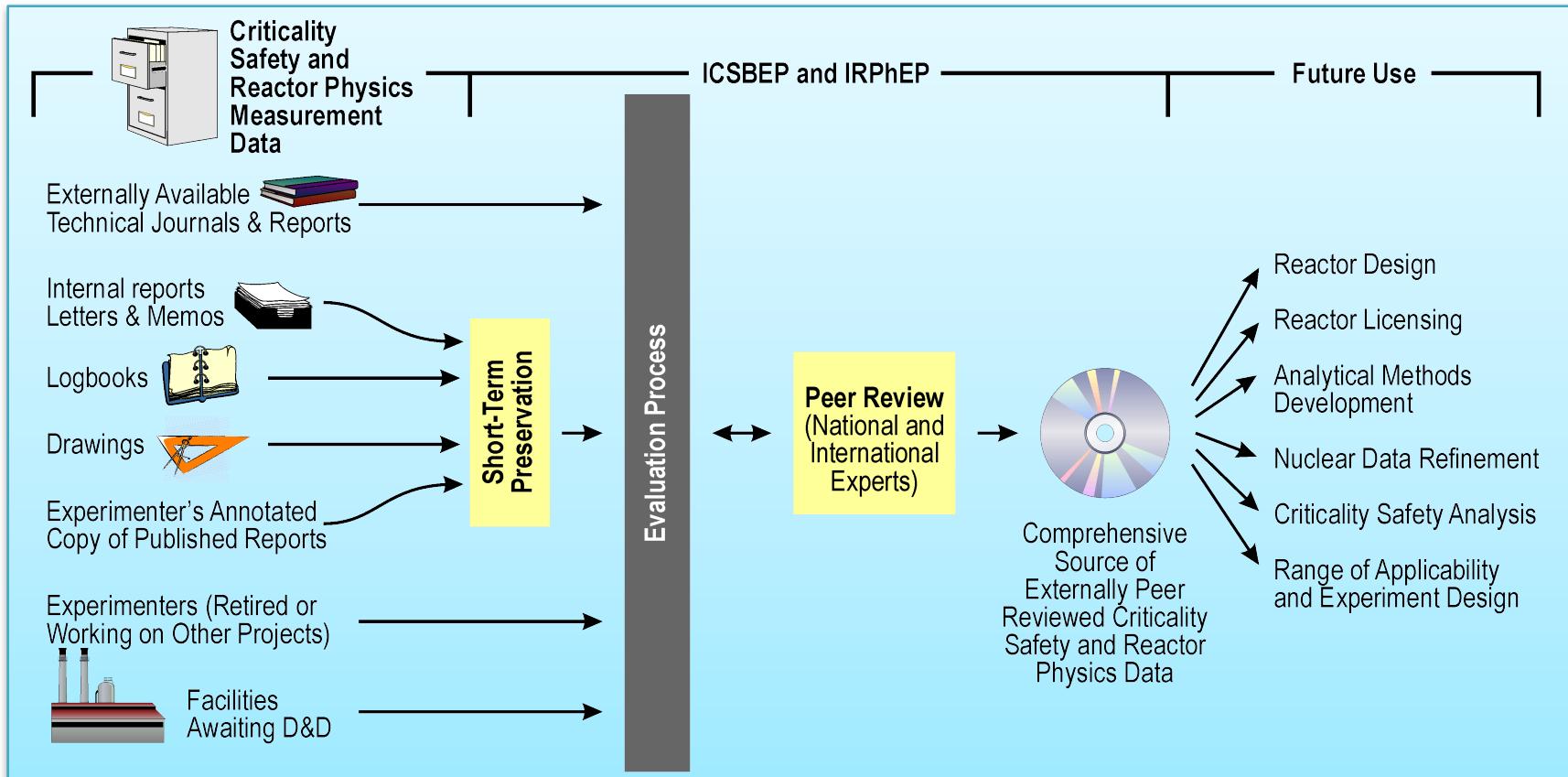


# Purpose

- Re-evaluate experiments of GCFR Core designs performed in the 1970s at the PROTEUS reactor.
- Preserve decades old integral nuclear data.
- Enable the validation of thorium, neptunium, plutonium, and uranium cross sections.



# Purpose of the ICSBEP and IRPhEP

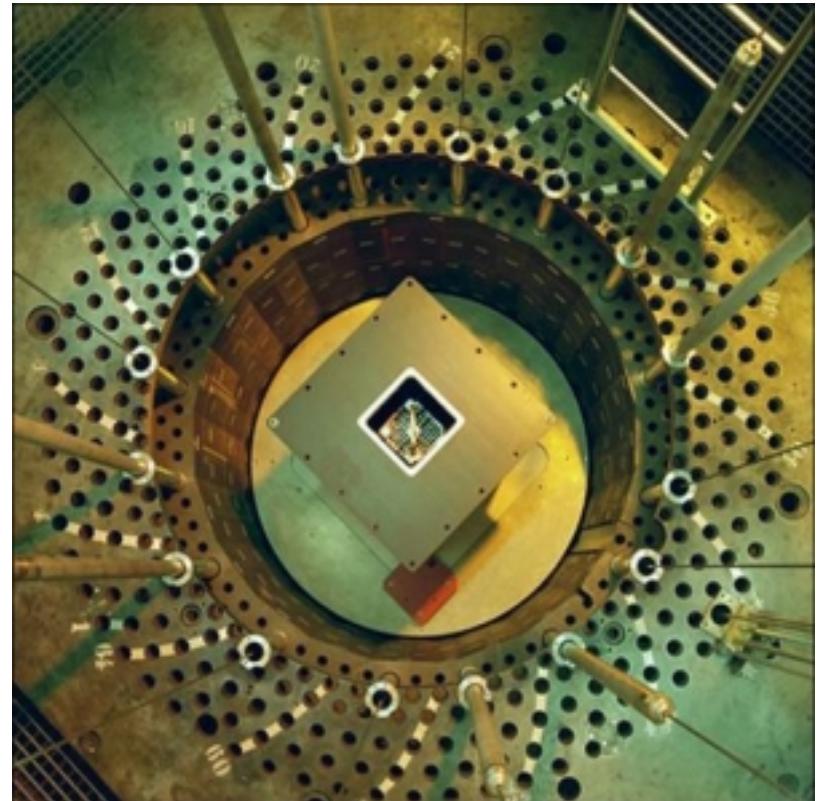


# Benchmark Process General Overview

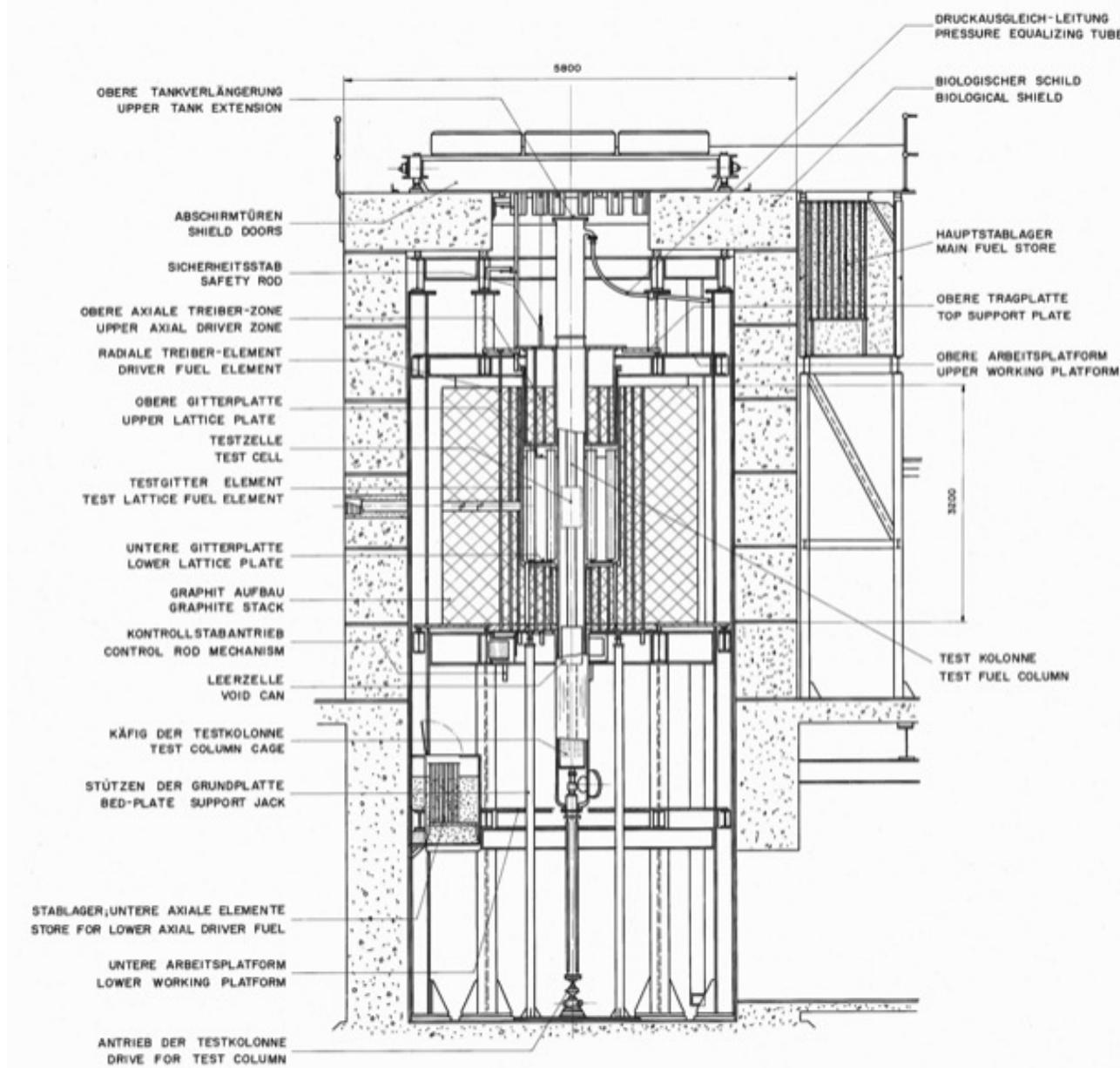
1. Identify Experiment
2. Evaluate Experiment
  - a. Prepare Benchmark Report
3. Internal Review of Benchmark Report
4. Submit Benchmark Experiment to ICSBEP/IRPhEP
5. Independent Review of Benchmark Report
6. Distribution of Benchmark Report to Technical Review Group
7. Technical Review Meeting
8. Resolve Action Items
9. Handbook Publication

# PROTEUS

- Lifetime: February 1968- April 2011
- Thermal Power Rating: 1 kilowatt.
- Driver Region Dimension: 3.5m diameter/ 3.5m height.
- Driver Fuel: 1m length UO<sub>2</sub> at 5 w/o.
- Central Cavity Dimension: 1.2m diameter/10m height.
- Central Cavity Fuel: Variable



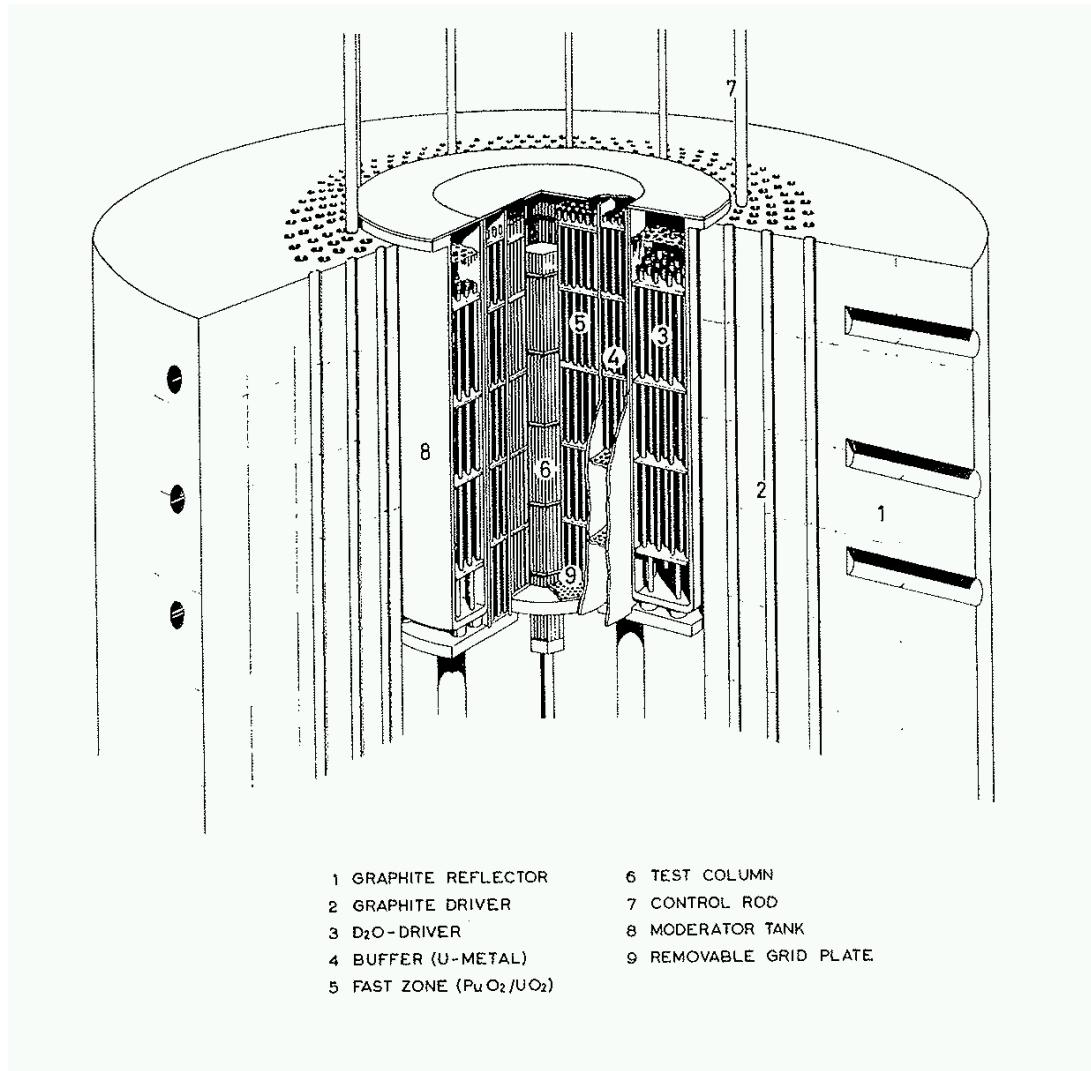
# PROTEUS



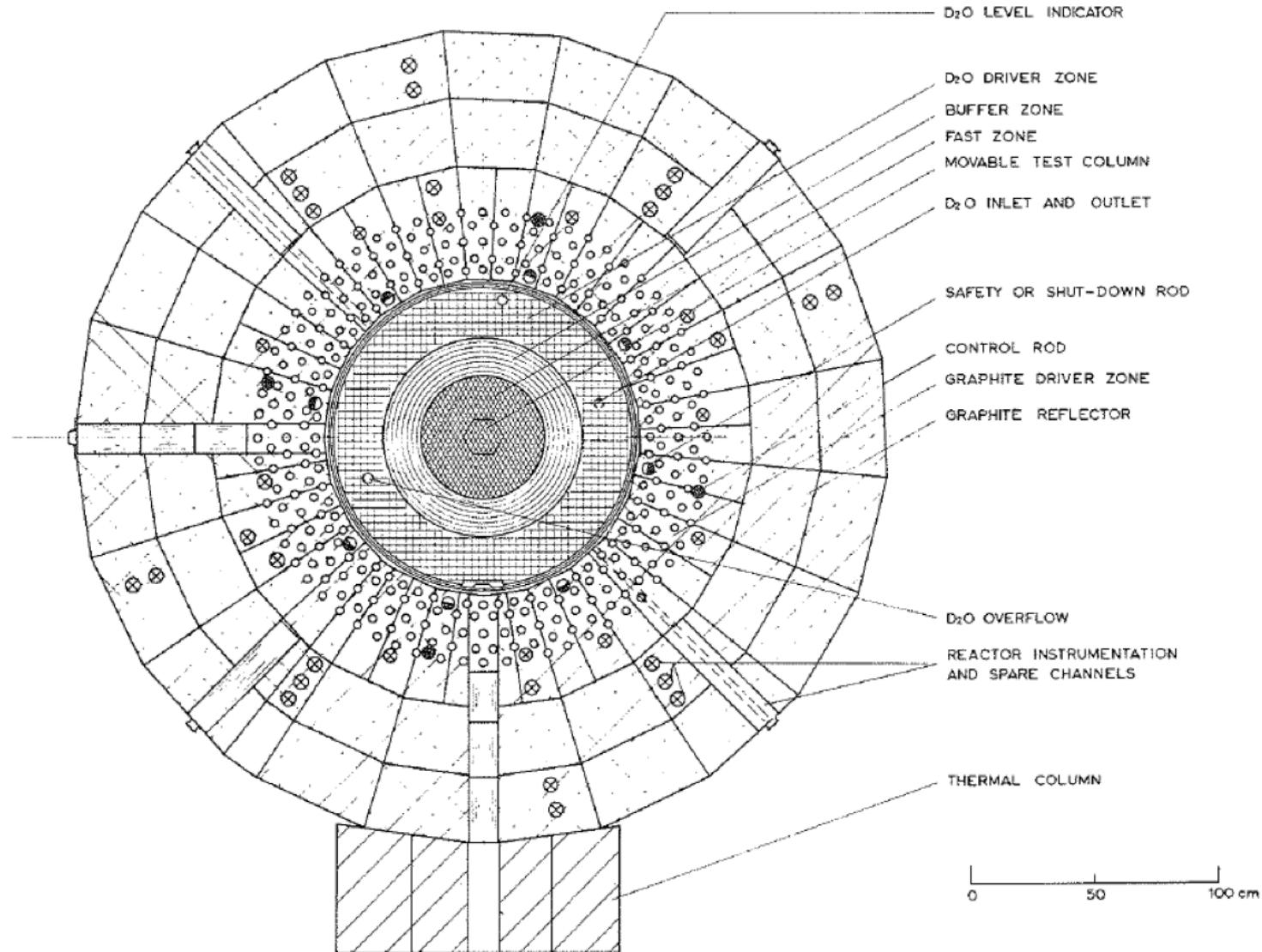
# GCFR-PROTEUS

- April 1972 to April 1979 – Benchmark Experiments in PROTEUS to validate Neutronics calculations in GCFR core
- Increased interest in GFR concepts & Th<sup>232</sup>/U<sup>233</sup> fuel cycle in the 70's
- Experiments required for accurate measurement of basic neutron cross section data, supplementing design calculations
- Initial cores – MOX cores (U238/Pu239 fuel cycle)
- Latter cores dedicated to Th232/U233 fuel cycle, homogeneous & heterogeneous distributions of Thorium
- C8/F9, F8/F9, C2/F9 etc, characterised breeding ratio, power distribution, neutron spectrum, axial & radial profiles etc.
- Experimental results compared with deterministic calculations

# Proteus GCFR Configuration

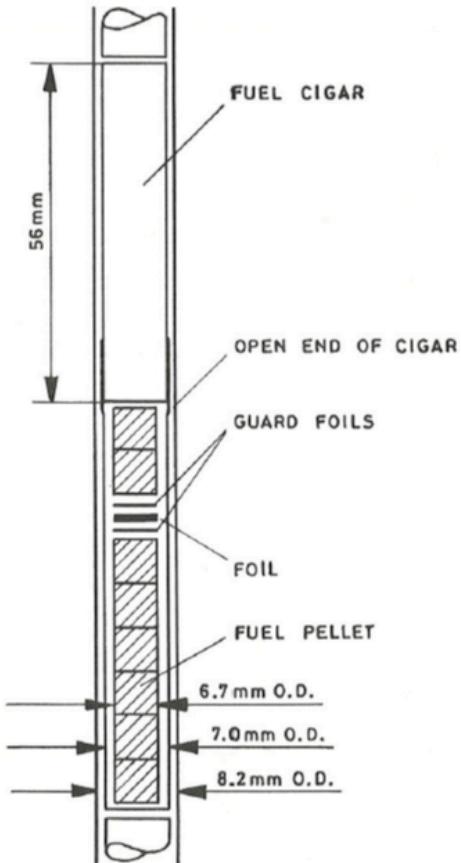


# Proteus GCFR Configuration



# Measurement Techniques

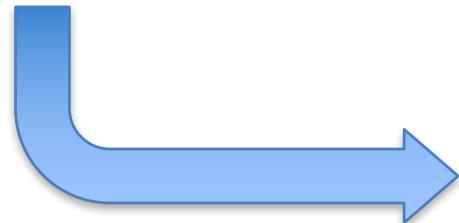
- Spectral Indices: Foil activation / fission chambers
- $\gamma$ -counting : Twin Ge(Li) or NaI detectors



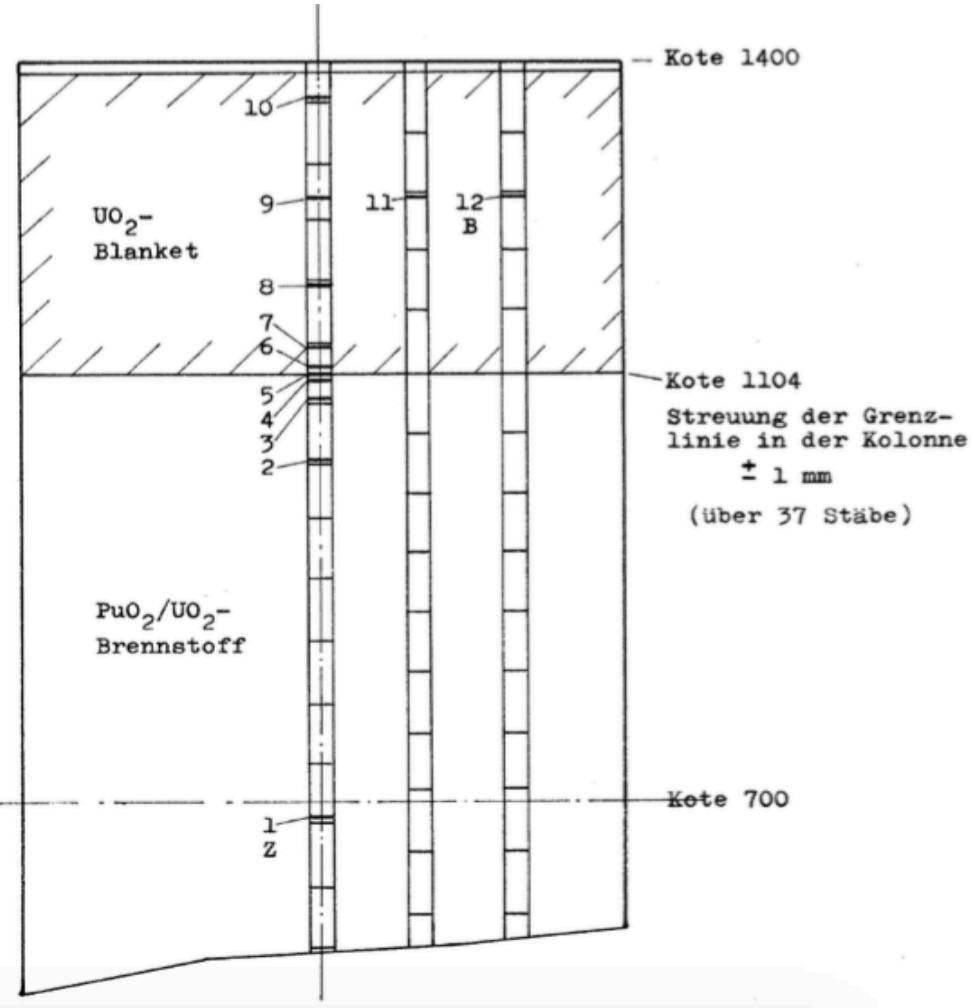
# Foil Measurements

## Experiment Values and Uncertainties

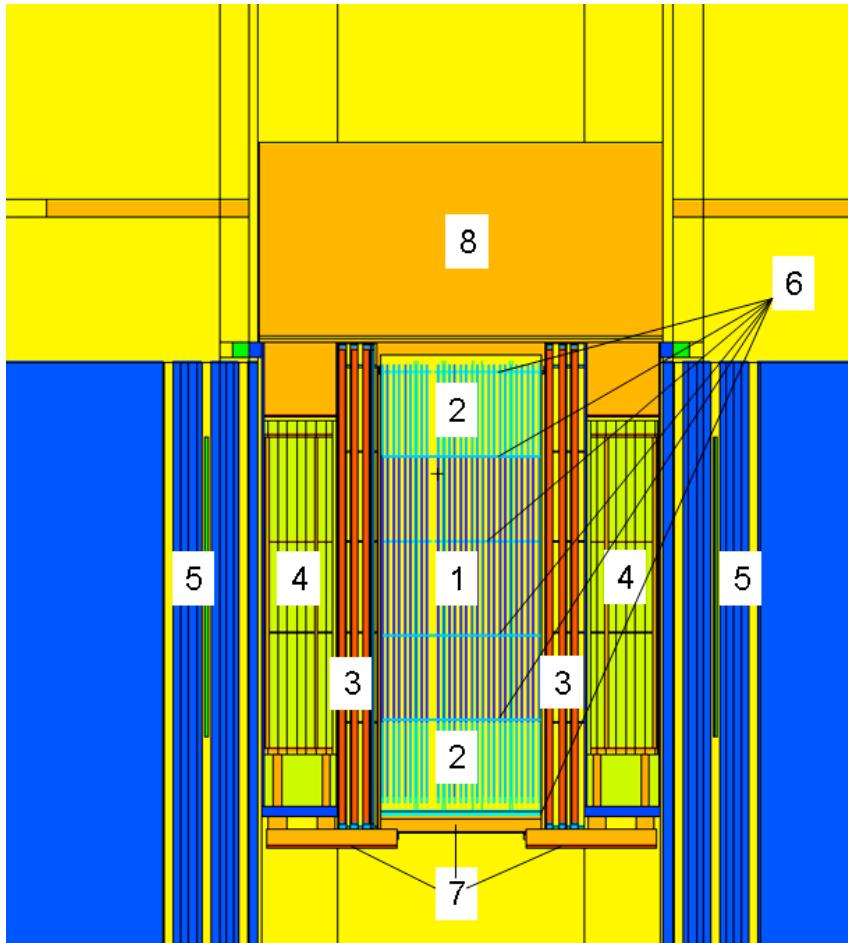
Ratio	Experiment	$1\sigma$
c8/f9	1.33E-01	1.463E-03 (1.1%)
f8/f9	3.11E-02	4.043E-04 (1.3%)
f5/f9	1.01E+00	1.414E-02 (1.4%)
c2/f9	2.00E-01	2.600E-03 (1.3%)
f2/f9	8.06E-03	1.612E-04 (2.0%)
f3/f9	1.52E+00	1.976E-02 (1.3%)
(n,2n)2/c2	6.84E-03	1.710E-04 (2.5%)
c7/f9	8.26E-01	1.8998E-02 (2.3%)
f7/f9	2.27E-01	4.086E-03 (1.8%)



Foil Locations

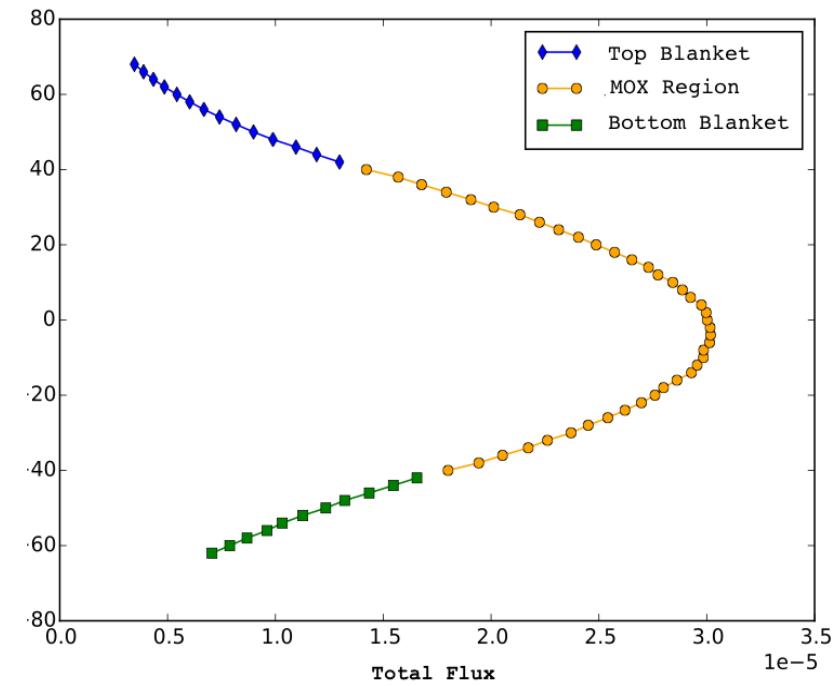
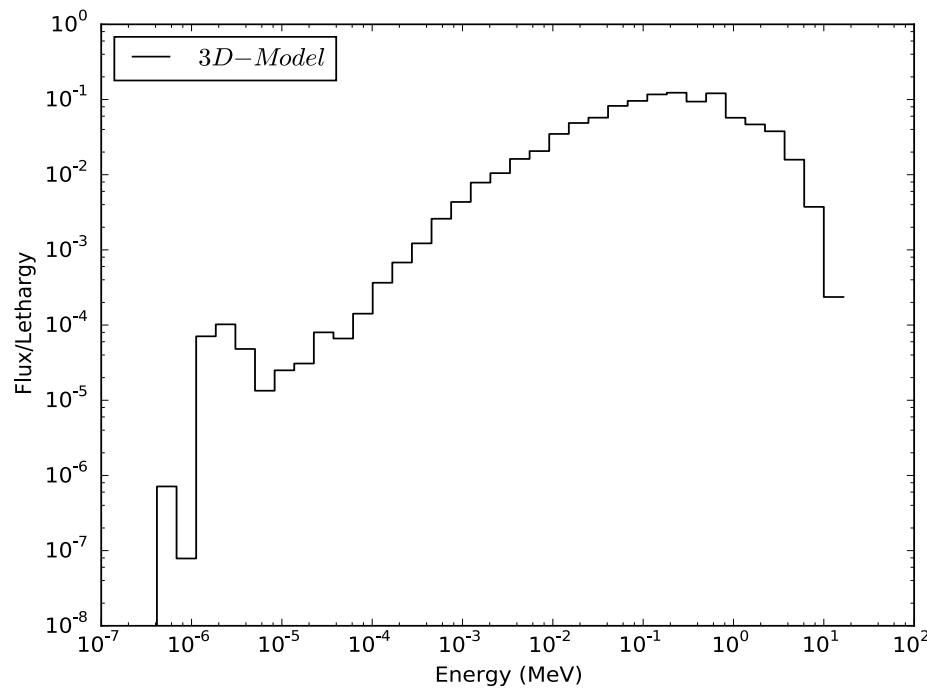


# Core 11 MCNP Model



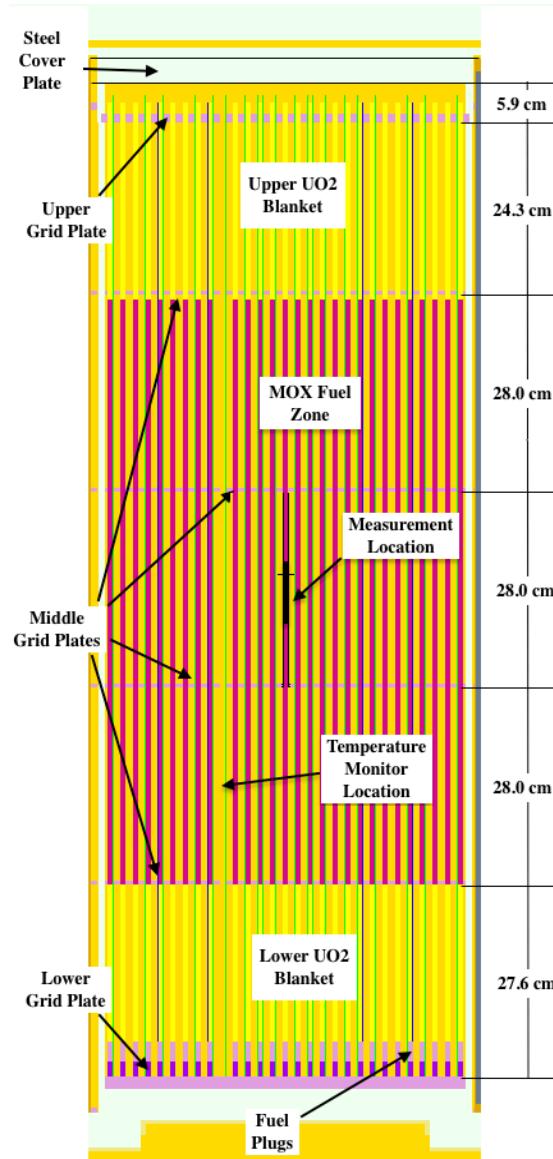
- 1 - Test Zone (MOX lattice)
- 2 - Blanket Zone (UO<sub>2</sub> lattice)
- 3 - Buffer zone
- 4 - D<sub>2</sub>O zone
- 5 - Graphite Driver Zone
- 6 - Grid plates
- 7 - Reactor support plates
- 8 - Steel Shielding

# Core 11 Energy Profile & Axial Flux Distribution



# Model Techniques

- Volume Averaging
- $RR = F4 * XS$
- $Spectral\ Index = \frac{RR_x}{RR_y}$
- Ratio allows for direct comparison between foil and model volumes



# Summary of experimental uncertainties in GCFR-PROTEUS Core 11

Spectral Indices	Measurement Uncertainty ( $\Delta/\text{MEAN}$ )	Evaluated Experimental Uncertainty ( $\Delta/\text{MEAN}$ )	Total Experimental Uncertainty ( $\Delta/\text{MEAN}$ )
c8/f9	1.4630E-03 (1.100%)	1.476E-04 (0.112%)	1.470E-03 (1.106%)
f8/f9	4.0430E-04 (1.300%)	7.427E-05 (0.235%)	4.111E-04 (1.322%)
f5/f9	1.4140E-02 (1.400%)	6.774E-04 (0.066%)	1.416E-02 (1.402%)
c2/f9	2.6000E-03 (1.300%)	1.058E-03 (0.514%)	2.807E-03 (1.404%)
f2/f9	1.6120E-04 (2.000%)	1.855E-05 (0.244%)	1.623E-04 (2.013%)
f3/f9	1.9760E-02 (1.300%)	1.288E-03 (0.086%)	1.980E-02 (1.303%)
(n,2n)2/c2	1.7100E-04 (2.500%)	1.018E-04 (1.333%)	1.990E-04 (2.910%)
c7/f9	1.8998E-02 (2.300%)	1.321E-03 (0.160%)	1.904E-02 (2.306%)
f7/f9	4.0860E-03 (1.800%)	3.072E-04 (0.135%)	4.098E-03 (1.805%)

## Perturbations

### Type:

Density, isotope fraction,  
composition, impurities

### Scaling Factors:

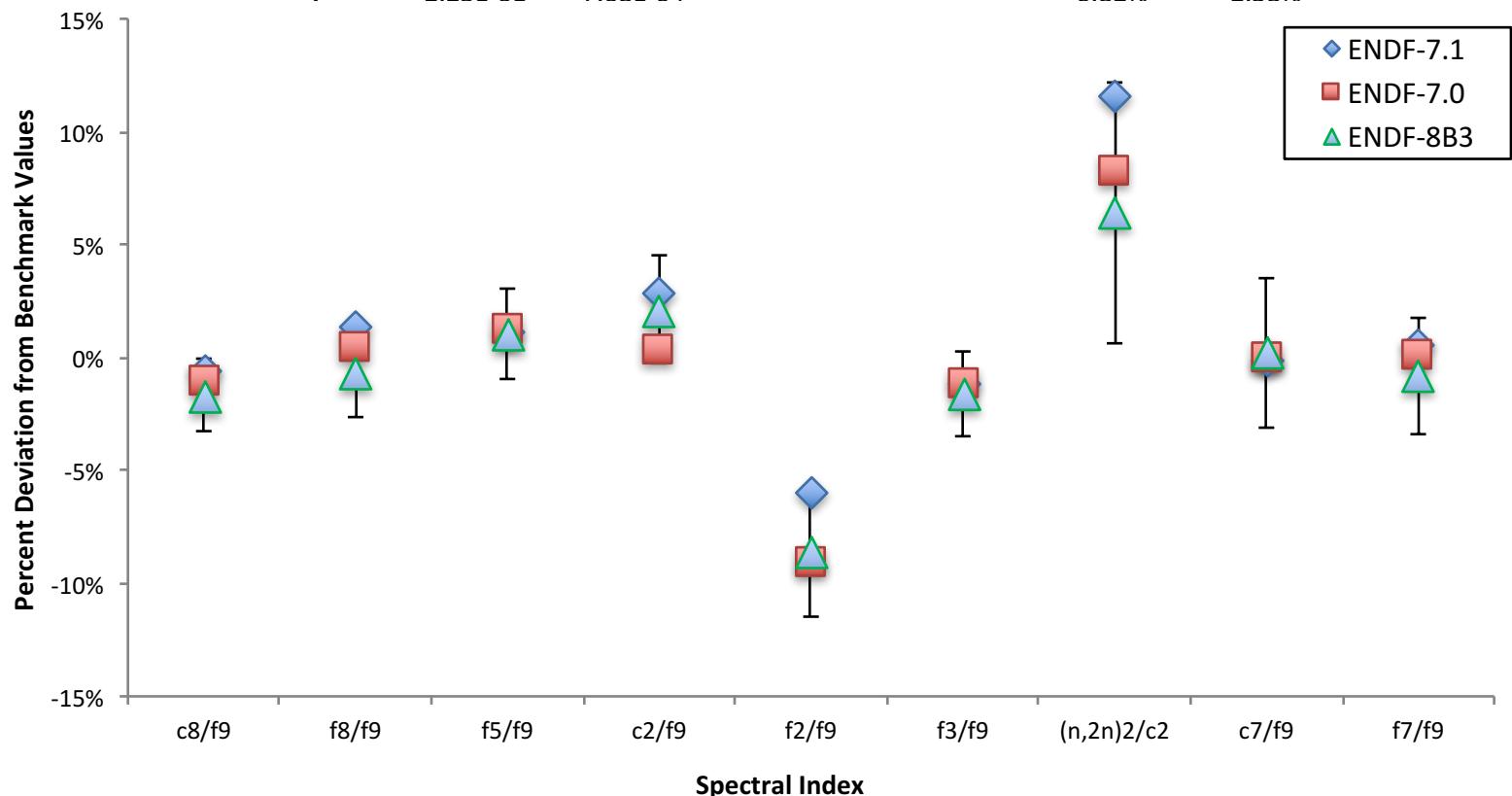
MCNP limitations  
10 to 100

### Materials:

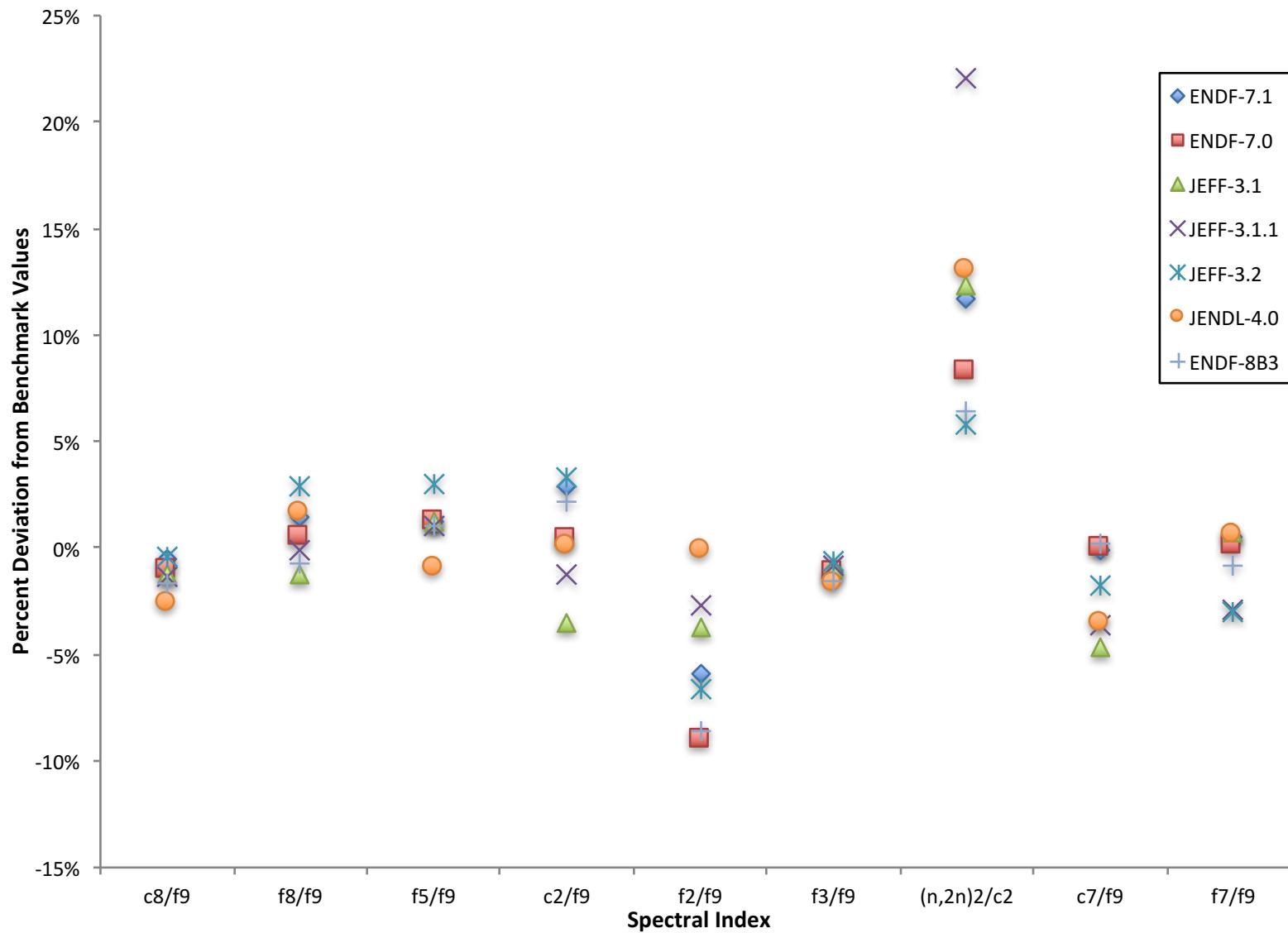
Fuel, structural materials  
(steel/aluminum), D<sub>2</sub>O,  
graphite

# Comparison of ENDF Cross Sections Core 11

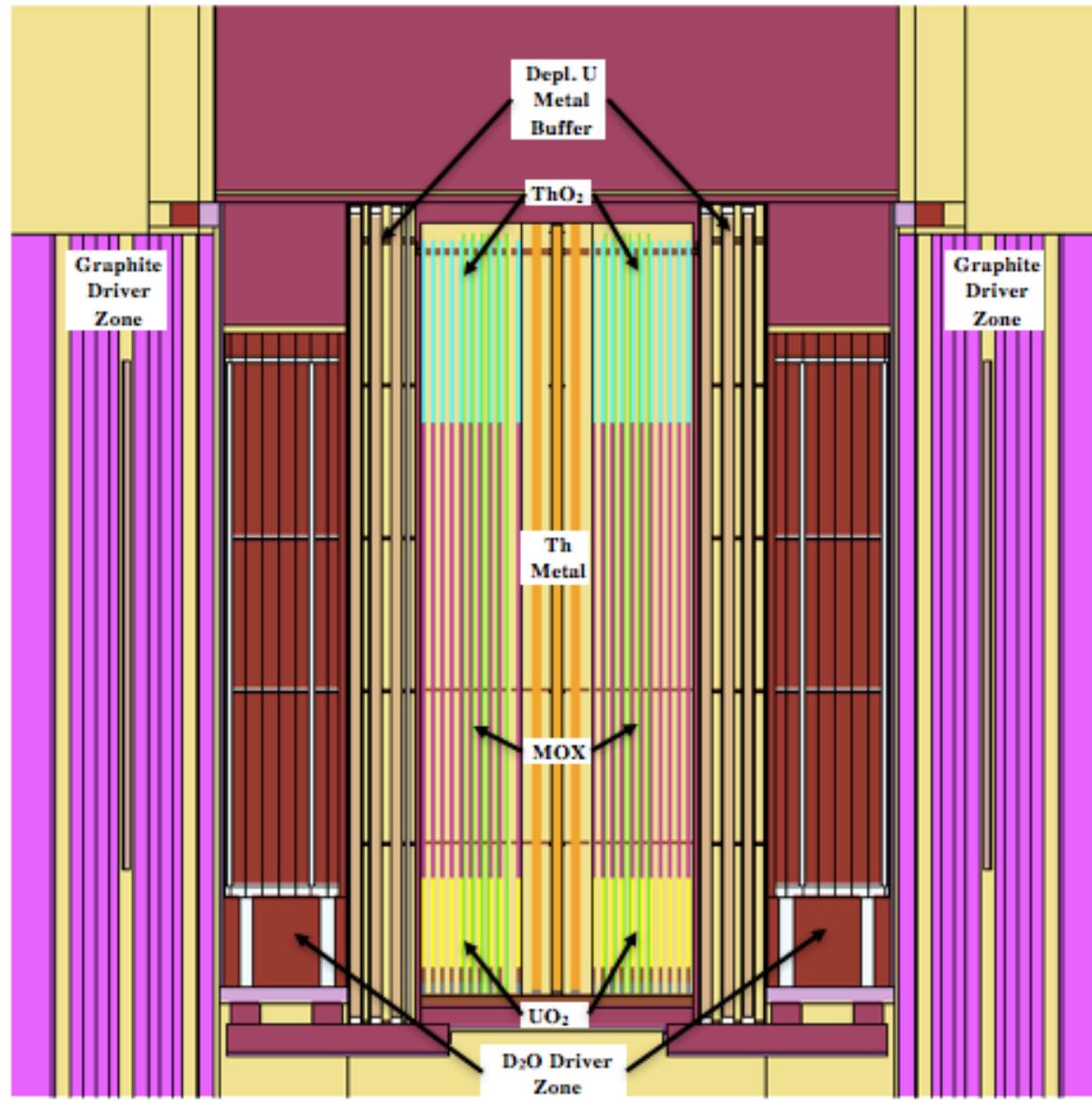
Spectral Indices	Calculated (ENDF-8B3)		Benchmark		(C/E)-1	
	Value	$\sigma$	Value	$\sigma$	Value	$\sigma$
c8/f9	1.31E-01	4.69E-04	1.330E-01	1.470E-03	-1.65%	1.60%
f8/f9	3.09E-02	1.64E-04	3.110E-02	4.111E-04	-0.69%	1.94%
f5/f9	1.02E+00	2.67E-03	1.010E+00	1.416E-02	1.05%	2.00%
c2/f9	2.04E-01	2.72E-03	2.000E-01	2.807E-03	2.14%	2.39%
f2/f9	7.37E-03	4.12E-05	8.060E-03	1.623E-04	-8.57%	2.90%
f3/f9	1.50E+00	5.36E-03	1.520E+00	1.980E-02	-1.61%	1.88%
(n,2n)2/c2	7.28E-03	2.95E-04	6.840E-03	1.990E-04	6.41%	5.77%
c7/f9	8.28E-01	4.71E-03	8.260E-01	1.904E-02	0.21%	3.31%
f7/f9	2.25E-01	7.68E-04	2.270E-01	4.098E-03	-0.82%	2.58%



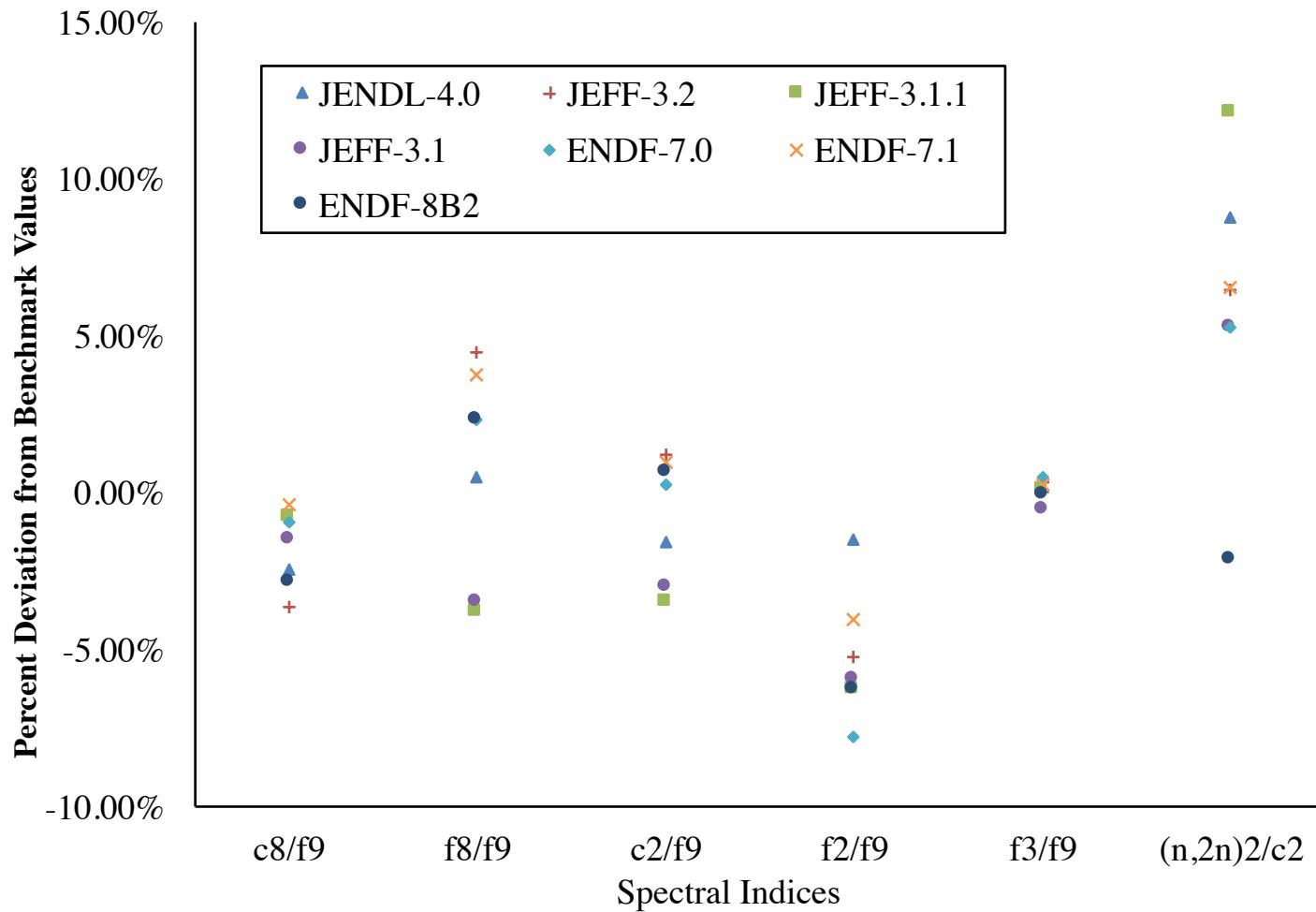
# Comparison of Cross Sections Core 11



# Core 15 Model



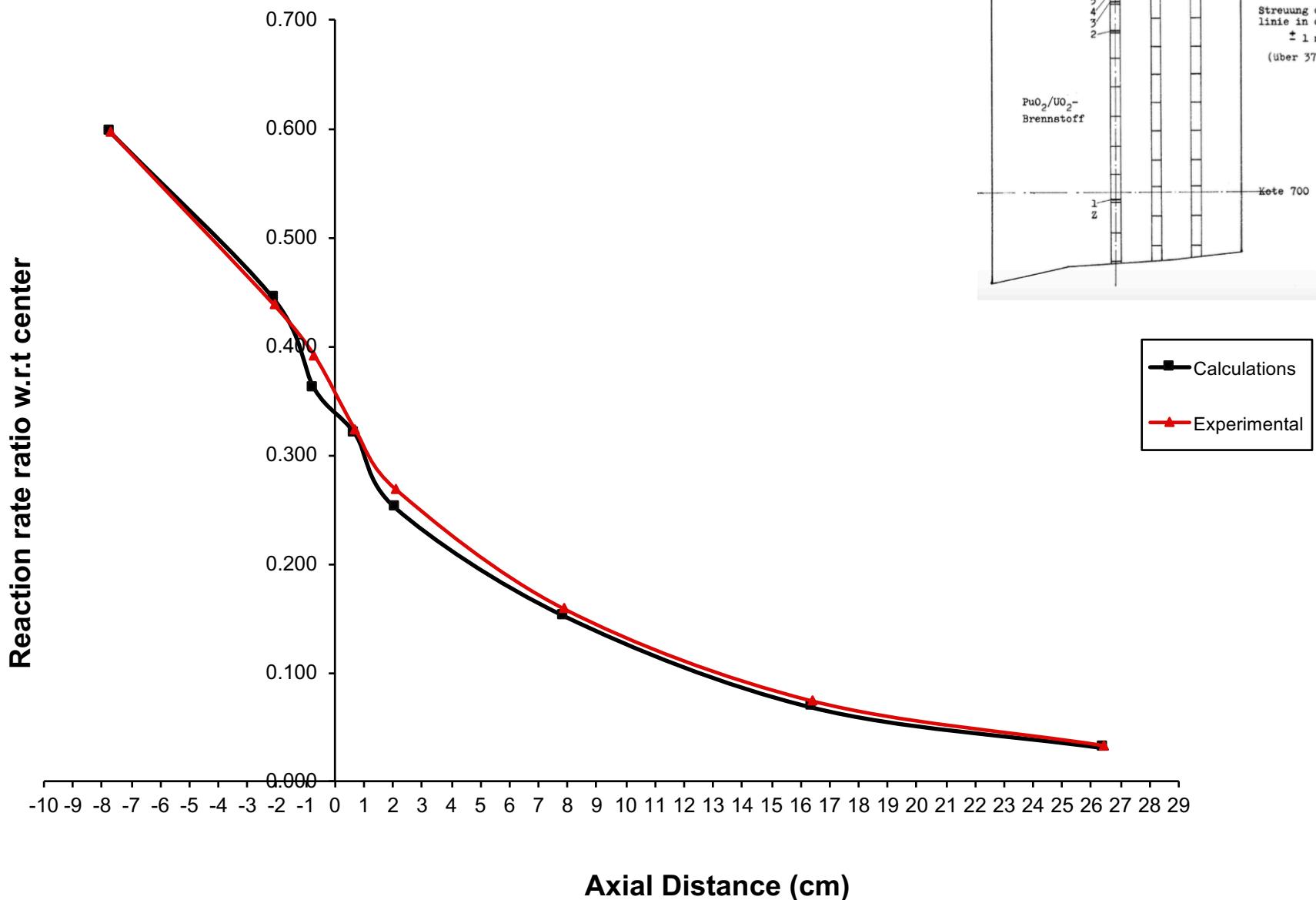
# Comparison of Cross Sections



# Questions?

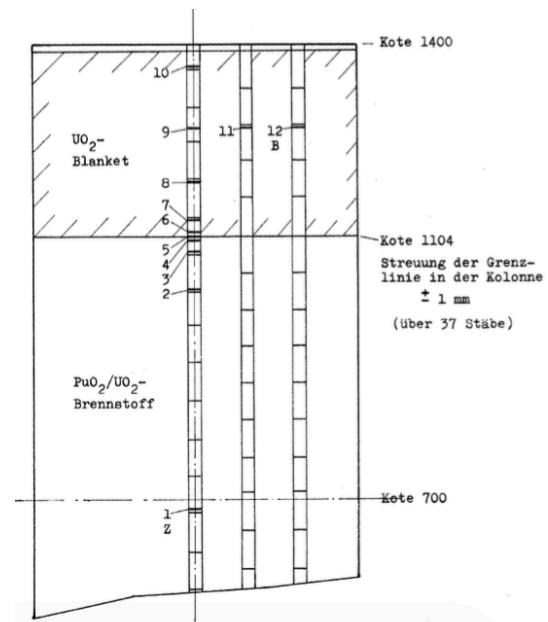
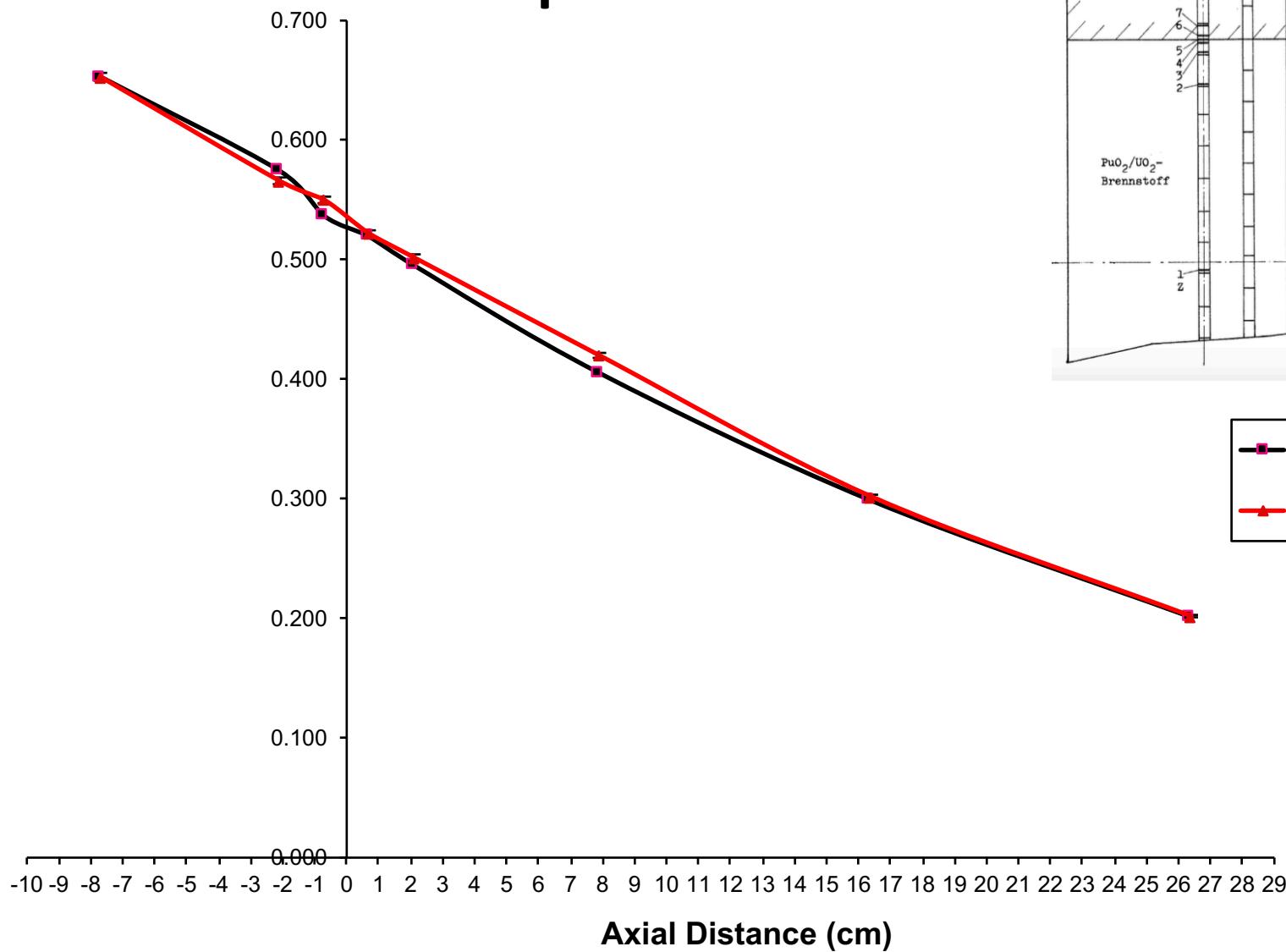


# Fission U238



# Capture U238

Reaction rate ratio w.r.t center



■ Calculations  
▲ Experimental

# Fission Pu239

